

# Separatrix ion to electron temperature ratio in the TCV and ASDEX Upgrade tokamaks

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## Abstract

Increased focus has been recently given to the ion temperature  $T_i$  near the separatrix and in the scrape-off layer (SOL) due to its impact on plasma confinement, wall sputtering, heat fluxes, and edge turbulence. In current tokamaks, high pedestal collisionality often clamps pedestal top ion and electron temperatures together. However, thermal decoupling occurs in the SOL due to the different ion and electron masses, leading to  $T_i/T_e > 1$  at the separatrix and in the SOL. Despite consistently observing elevated  $T_i$  relative to  $T_e$  near the separatrix in various tokamaks, a comprehensive study remains missing due to the measurement complexity. This study, for the first time, explores the primary dependencies of  $T_i/T_e$  at the separatrix, focusing on the variation of electron density and connection length to scan SOL collisionalities, and the variation of input power to scan ion and electron heat fluxes, in the ASDEX Upgrade and TCV tokamaks. Surprisingly,  $T_i/T_e$  shows weak or no dependence on electron density and connection length, despite their expected role in the SOL coupling. However, the  $T_i/T_e$  ratio is strongly influenced by ion and electron heating levels in the plasma core. This suggests that the upstream  $T_i/T_e$  is primarily regulated by relative ion and electron heat fluxes from the core, with coupling playing a minor role.

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(Some figures may appear in colour only in the online journal)

## 1. Introduction

Recently, the importance of the ion temperature  $T_i$  close to the separatrix and in the scrape-off layer (SOL) has been given increased attention due to its impact on a range of areas in tokamak physics. This includes determining plasma confinement [1] and understanding plasma–wall interaction processes such as physical sputtering, reflection, and impurity release [2]. Furthermore, the relationship between  $T_i$  and the electron temperature  $T_e$  is critical in predicting heat fluxes in the SOL [3, 4] and explaining turbulence in the edge region [5]. Therefore, an accurate comprehension of the behavior and dependencies of  $T_i$  near the separatrix is essential for setting the conditions and boundaries for various aspects of fusion plasma physics.

In present tokamaks, the pedestal collisionality is generally so high that the pedestal  $T_i$  and  $T_e$  are clamped together. Nonetheless, in the SOL, the thermal confinement of ions and electrons is short compared to their equilibration time, leading to thermal decoupling. According to the Spitzer–Härm model for heat conduction, the expression for the parallel heat flux of electrons and ions is given by:

$$q_{SH(e,i)} = \kappa_{0(e,i)} T_{(e,i)}^{5/2} \frac{dT_{(e,i)}}{dl} \quad (1)$$

where  $\kappa_{0(e,i)}$  denotes the thermal conductivity for electrons or ions, and  $l$  is the coordinate along the field lines. In deuterium plasmas, the thermal conductivity values are approximately  $2000 \text{ WeV}^{-7/2} \text{ m}^{-1}$  for electrons and  $60 \text{ WeV}^{-7/2} \text{ m}^{-1}$ , for ions, as reported in [7]. The difference in thermal conductivity results in ions having a higher temperature than electrons, a phenomenon consistently supported by measurements [6, 8, 9]. Following the approach of [6], integrating the relation (1) as usual from the upstream region (' $u$ ') to the target (' $t$ '), assuming  $T_{u,(e,i)}^{7/2} \gg T_{t,(e,i)}^{7/2}$ , and taking the ratio between  $T_{u,i}$  and  $T_{u,e}$ , yields to the following expression:

$$\frac{Q_i}{Q_e} = \frac{\kappa_{0,i}}{\kappa_{0,e}} \left( \frac{T_{u,i}}{T_{u,e}} \right)^{7/2} \quad (2)$$

where  $Q_i$  and  $Q_e$  are the integrated ion and electron heat flux crossing the separatrix. This relation directly connects the ratio of total ion and electron heat flux to the upstream temperature ratio, as shown in blue in figure 1. Notably, for  $Q_i/Q_e = 1$ , Spitzer–Härm predicts  $T_i/T_e \approx 2.7$ . This suggests that observing a significantly higher  $T_i$  compared to  $T_e$  upstream is natural and driven by parallel transport in the SOL. However, Spitzer–Härm is a fluid description typically applicable in regimes where power enters the flux tube from the upstream end, while the particle source is concentrated near

the target such that convection is negligible, i.e. in the high recycling regime. Such conditions are established at relatively high SOL collisionality, i.e. low temperature and/or high density. However, being a fluid model, Spitzer–Härm implicitly assumes that the electron or ion mean free path is short compared to any other relevant scale length (system or gradient scale lengths). When this assumption breaks down, accounting for kinetic effects becomes necessary. In practice, they are heuristically modeled in codes like SOLPS-ITER [10] or EMC3-EIRENE [11] through a heat flux limiter, an additional term in the parallel heat conduction that is implemented as:

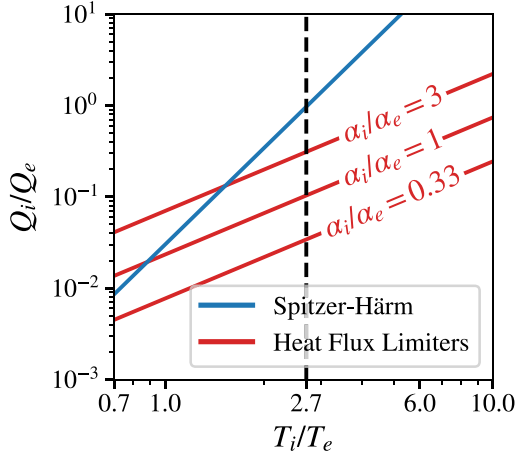
$$q_{FS(e,i)} = \alpha_{(e,i)} n v_{th(e,i)} k_B T_{(e,i)} \quad (3)$$

where  $\alpha_{(e,i)}$  is a heuristic parameter called heat flux limiting factor, and  $v_{th}$  the thermal velocity. In fluid codes, the total parallel heat transport is implemented simply as the harmonic average of the Spitzer–Härm and free-streaming heat fluxes and it effectively aims to limit the heat flux a fraction of the free-streaming value ( $q_{FS(e,i)} = n v_{th(e,i)} k_B T_{(e,i)}$ ). As before, integrating the relation (3) in the SOL, yields to:

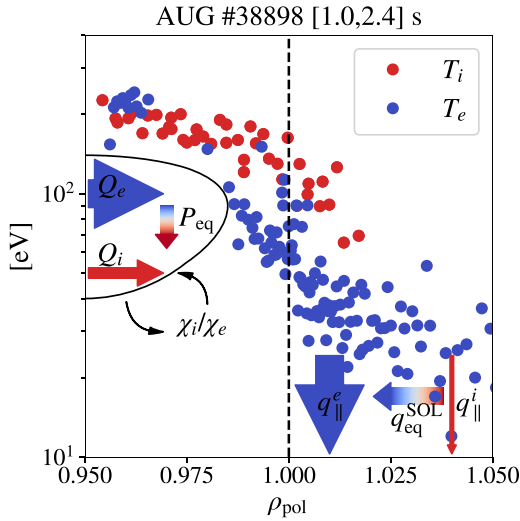
$$\frac{Q_i}{Q_e} = \frac{\alpha_i}{\alpha_e} \left( \frac{m_e}{m_i} \right)^{1/2} \left( \frac{T_{u,i}}{T_{u,e}} \right)^{3/2} \quad (4)$$

which again links the ion and electron heat fluxes to the temperature ratios. Note the weaker dependence on the temperature ratio, which allows for a much higher upstream temperature ratio compared to Spitzer–Härm conduction. In the electron case,  $\alpha_e$  has been calibrated against SOL kinetic simulations [12] and is typically set to 0.3 [13, 14]. In contrast,  $\alpha_i$  remains largely unknown and is often assumed to be equal to  $\alpha_e$ . Figure 1 illustrates the relation (4) in red for different values of  $\alpha_i/\alpha_e$ , emphasizing the importance of experimentally linking heat fluxes to the temperature ratio. Establishing this connection would be crucial for ensuring the self-consistency of SOL simulations and for cross-validating turbulence codes. This analysis is conducted for the first time in this work, and presented in section 2.3.

All the previous consideration do not include any heat exchange between ion and electrons. While this might seem like an oversight from a core transport perspective, it is important to recall that we are comparing parallel transport, which is obviously much faster than typical perpendicular transport time scales, to equilibration time scales. Stangeby in [7] highlights the importance of heat exchange as being of great importance and propose a model for  $T_i/T_e$ , supported by analysis of SOLPS modeling results. As the upstream equipartition between ions and electrons  $q_{eq}^{SOL}$  is generally much larger than the parallel ion heat flux  $q_{\parallel}^i$ , the ion heat is mostly



**Figure 1.** Relation between the integrated ion-to-electron heat flux ratio across the separatrix ( $Q_i/Q_e$ ) and the upstream ion-to-electron temperature ratio ( $T_i/T_e$ ), shown for the Spitzer–Härm limit in blue and at three different heat flux limiter ratios in red (Reproduced from [6]. © IOP Publishing Ltd All rights reserved.)



**Figure 2.** Schematic of the key physics mechanisms regulating the ion (red) to electron (blue) temperature ratio in the vicinity of the separatrix.

exhausted through the coupling to the electron heat. As a result, the ion to electron temperature ratio scales like the SOL collisionality:

$$\nu_{\text{SOL}} \sim n_{e,u} \cdot \frac{L}{T_i^{5/4} \cdot T_e^{3/4}} \quad (5)$$

where  $n_{e,u}$  is the upstream electron density and  $L$  the connection length. Moreover, the absolute value of  $T_i$  and  $T_e$  also depends on the relative strength of the local  $q_{\text{eq}}^{\text{SOL}}$  compared to the total perpendicular ion and electron heat flux,  $Q_i$  and  $Q_e$  respectively, entering in the SOL. Stangeby derived a scaling of the ion to electron temperature ratio based only on JET modeling results obtaining  $T_i/T_e \sim ((Q_i/Q_e)/\nu_{\text{SOL}}^*)^{2/7}$ . Additionally, the ion–electron coupling plays a role within the confined plasma as well, influencing  $Q_i$  and  $Q_e$  through the

equipartition term  $P_{\text{eq}}$ , which is calculated as:

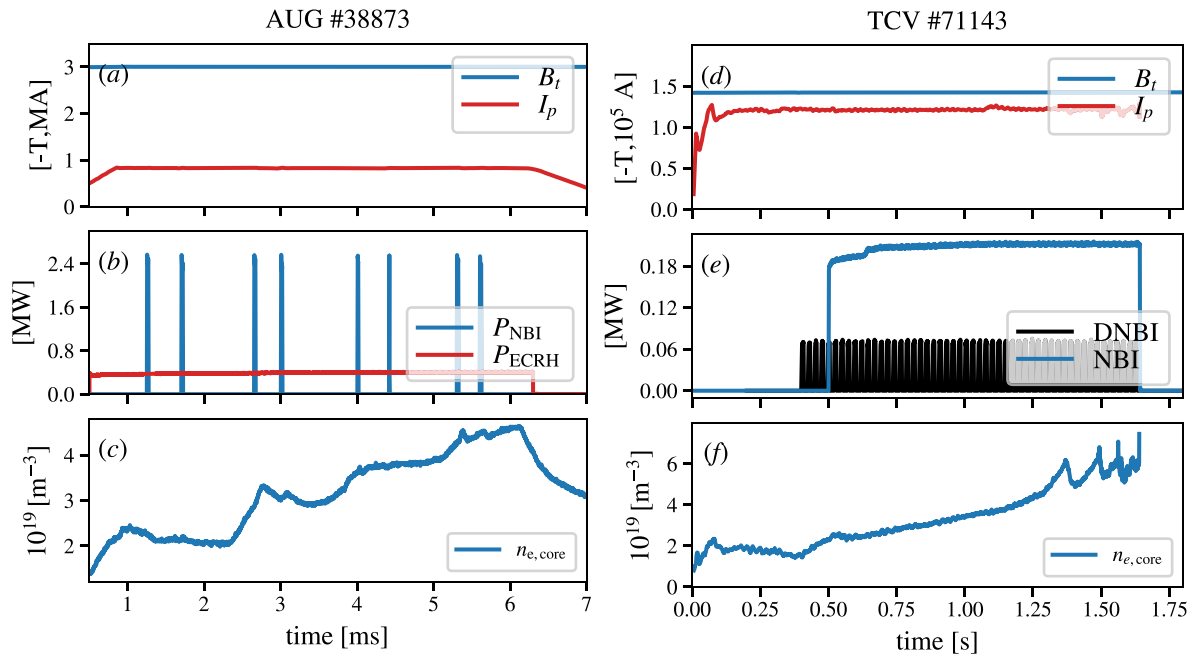
$$P_{\text{eq}} \approx \nu_{ie} (T_e - T_i) n_e V \sim \frac{T_e - T_i}{T_e^{3/2}} n_e V \quad (6)$$

where  $V$  is the plasma volume and  $\nu_{ie}$  is the ion–electron collision frequency. Finally,  $T_i/T_e$  is influenced also by the ratio of the ion to the electron heat transport, i.e.  $\chi_i/\chi_e$ , which depends on the underlying turbulence and, hence, in turn by  $T_i/T_e$  itself [5, 15]. This complex feedback mechanism is visually summarized in figure 2, together with some example  $T_i$  (red) and  $T_e$  (blue) profiles from the ASDEX Upgrade (AUG) tokamak. The reader should note that the assumption made in figure 2 that  $Q_e > Q_i$  is for illustration purposes only and does not reflect any general condition.

Although an elevated ion temperature  $T_i$  relative to the electron temperature  $T_e$  near the separatrix has been consistently observed in various tokamaks, a comprehensive investigation is still lacking due to the complexity of the measurements involved [6, 16]. In this work, we report, for the first time, a study of key dependencies of  $T_e/T_i$  at the separatrix, namely  $n_e$ ,  $L$  via magnetic field ( $B_i$ ) and plasma current ( $I_p$ ), and input power in the ASDEX Upgrade and TCV tokamak. In particular, to possibly enhance the effect of the ion–electron coupling relative to the total heat fluxes, only L-mode discharges were considered. Along with the absolute temperature values, we also present first calculation of the heat fluxes.

## 2. Experiments and analysis

The experimental strategy to obtain accurate separatrix  $T_i/T_e$  ratios at the ASDEX Upgrade and TCV tokamak in L-mode is different due to the specific characteristics of the two devices and diagnostics involved in this study. In particular, at AUG, the edge and core ion temperature measurements are performed via Charge Exchange Recombination Spectroscopy (CXRS) on an heating beam [17, 18] limiting the possibility to obtain ion temperature data to only few beam blips as a long phase, typically of the order of 30 ms, of NBI heating would trigger the H-mode. Instead, at the TCV tokamak, the CXRS system is based on a low power ( $\approx 50$  kW), non-perturbing, diagnostic beam [19, 20] (DNBI) allowing continuous measurements during the entire discharge. It is important to note that the traditional charge exchange recombination diagnostic at AUG and TCV does not provide a direct measurement of the temperature of the main ions  $T_i$ , but instead measures the temperature of the impurity ions  $T_Z$  that are involved in the CX reaction with the beam [21]. Specifically, in the case of the experiments conducted on AUG, the temperature of the fully ionized boron ions was measured, while in the experiments at TCV, the temperature of the carbon ions. Recent advancements in main ion CX systems [22–24] have shown that  $T_i$  is generally lower than  $T_Z$ . However, in L-mode—the scenario of interest for this study—the difference at the separatrix remains within the error bars. Notably, a recent study on AUG [25] found no difference between  $T_i$  and  $T_Z$  at the separatrix. However, depending on the plasma scenario, deviations can occur toward the plasma core, with  $T_i > T_Z$ .



**Figure 3.** Plasma scenarios in AUG ((a) to (c)) and TCV ((d) to (f)) for studying the ion to the electron temperature ratio at the separatrix. (a), (d): Toroidal magnetic field (blue) and plasma current (red); (b), (e) NBI power (blue), ECRH power (red), and diagnostic NBI (black); (c), (f) core line averaged electron density.

**Table 1.** Parameter scan for AUG and TCV.

	$I_p$ [MA]	$B_t$ [T]	$P_{in}$ [MW]
AUG	0.8	-2.5, -3.0	Ohmic; ECRH: 0.4
TCV	0.24, 0.28, 0.32	-1.0, -1.2, -1.45	Ohmic; NBI: 0.2; ECRH: 1.3, 3.0, 4.6, 5.0

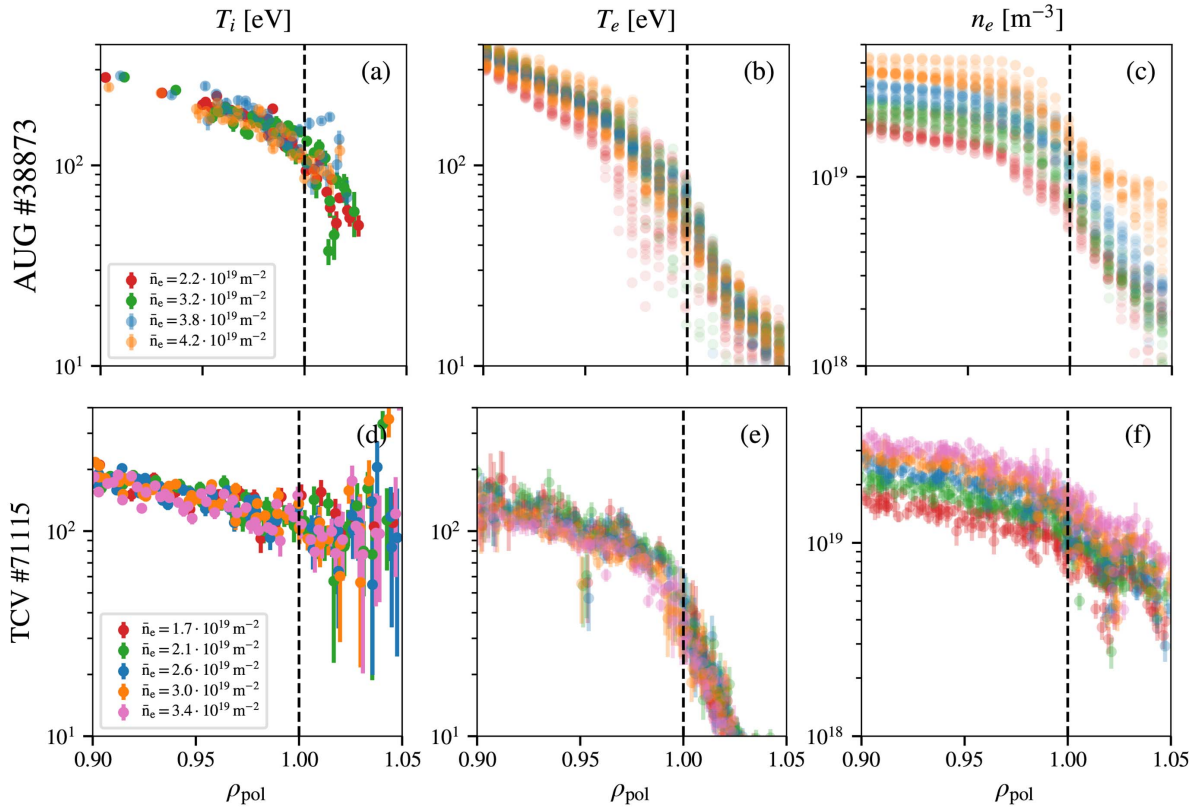
Regarding the electron temperature data, at AUG, the integrated data analysis diagnostic [26] was used, whereas at TCV, the Thomson scattering system [27, 28] was employed. As is traditionally done for determining the separatrix of plasma parameters and their gradients, a linear fit of the logarithmic profiles was performed near the separatrix, typically within a window of  $\rho_p \in [-0.98, 1.02]$ , to identify the separatrix values of the plasma. The fit was tested with different window sizes and found to be relatively robust, except when using excessively large windows. An exception to this is the  $T_i$  data from TCV, for which, as explained later in this section, only data within the separatrix were considered. Furthermore, no radial shifts were applied during profile mapping. It is important to note that this work focuses on the relative changes in  $T_i/T_e$  rather than its absolute values. This approach makes the comparison more robust against uncertainties related to equilibrium reconstruction and diagnostic positioning errors.

Figure 3 compares the ASDEX Upgrade (from figures 3(a)–(c)) and the TCV (from figures 3(d)–(f)) typical discharges used for this study. Figures 3(a) and (d) show the plasma current  $I_p$  in red and toroidal magnetic field  $B_t$  in blue. Figures 3(b) and (e) report the external heating sources: the NBI in blue, the electron cyclotron resonance heating

(ECRH) in red, and the DNB in black. Finally, figures 3(c) and (f) report the core electron density as it is stepped up in AUG and ramped up in TCV. All the discharges analyzed in this work are in the favorable  $B \times \nabla B$  configuration in order to have comparable SOL topologies. Moreover, in the case of AUG, the experiments have been performed right after boronization to increase the signal to noise ratio of the CXRS diagnostic. This type of discharge has been performed at different  $B_t$ ,  $I_p$  and input powers for the values reported in table 1. The ECRH power scan in TCV has been extracted from existing discharges performed at a constant density and is therefore different from the scenario reported in figure 3. In particular, for TCV, accessing the low  $B_t$  / high  $I_p$  regimes from the possible combinations in table 1 was not feasible, as it would quickly lead to the  $\beta$ -limit.

### 2.1. Collisionality scan

The most important parameter to vary in the  $\nu^*$  scan is the electron density (see equation (5)). Adjusting  $n_e$  results in the most significant change in  $\nu^*$ , reaching up to a factor of two. In contrast, variations in  $B_t$  and  $I_p$  within the machine limits affect the connection length, leading to changes in  $\nu^*$  of up



**Figure 4.** Edge ion temperature ((a), (d)), electron temperature ((b), (e)), and electron density ((c), (f)) for an L-mode density scan in AUG (from (a) to (c)) and TCV (from (d) to (f)). Different colors represent different electron densities.

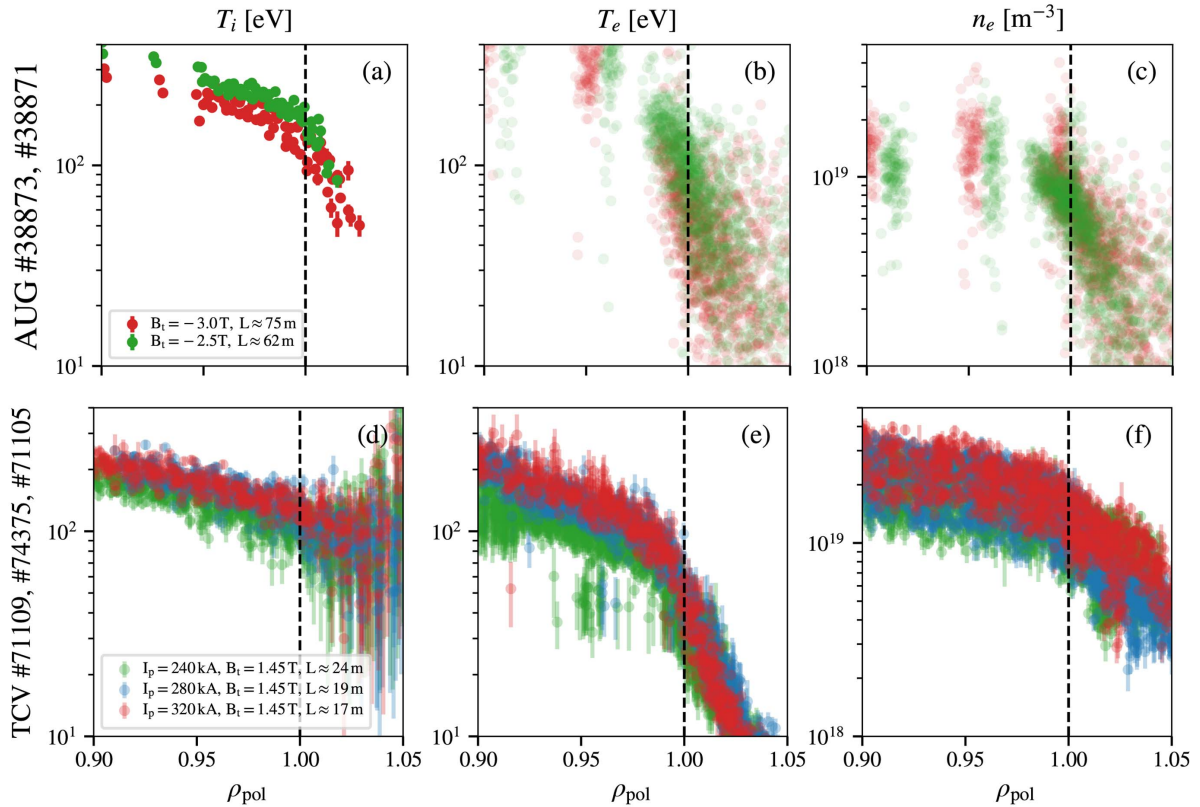
to 30%. In figure 4, the edge profiles of  $T_i$  (4(a)),  $T_e$  (4(b)), and  $n_e$  (4(c)) are shown for the AUG discharge depicted in figure 3. Different colors indicate different time points along the discharge. The most striking aspect of this scan is that, despite a roughly twofold increase in the separatrix density, both electron and ion temperatures remain unchanged, resulting in a constant  $T_i/T_e$ . This is in clear contrast with Stangeby's model for  $T_i/T_e$  at the separatrix. It is important to note that the range of  $\nu^*$  scanned in this discharge corresponds exactly to the regime where Stangeby reports a strong dependence of  $T_i/T_e$  on  $\nu^*$ , specifically from approximately  $\nu^* = 5 \times 10^{-16}$  to  $\nu^* = 1 \times 10^{-17}$  (see figure 4.20 in [7]). The averaged value of  $T_i/T_e$  over the whole scan is  $2.4 \pm 0.2$ , which is typical for AUG. However, as mentioned at the beginning of this section, the relative alignment between diagnostics is not straightforward, and uncertainties in the equilibrium may introduce errors in the estimation of  $T_i/T_e$ . What remains robust, since figures 4(a)–(c) show a single discharge, is the relative change of  $T_i/T_e$  with  $n_e$ , which is the primary purpose of this study.

Figures 4(d)–(f) show the edge profiles of  $T_i$ ,  $T_e$ , and  $n_e$ , respectively, for a density scan at TCV, with different colors indicating different core densities. In the case of TCV, the  $T_i$  profiles (figure 4(d)) exhibit a significant increase in scatter and error bars for  $\rho_{\text{pol}} > 1$ . This is a typical effect when the signal-to-noise ratio of the CX emission is very low. In such cases, the Gaussian fit still attempts to converge by fitting over the spectral region where the active and passive parts of the line are subtracted—a process that is never perfect. As a result, the

fit often converges to a broader Gaussian, leading to an overestimation of the temperature itself. Nevertheless, the profiles appear reliable up to  $\rho \approx 1$ , i.e. up to the region of interest. Here, consistent with the observations at AUG, no significant change of  $T_i$  or  $T_e$  is observed with varying  $n_e$  by roughly a factor of 2 (figure 4(e)). This further suggests that the coupling between ions and electrons may not be the dominant factor in determining the physics of  $T_i/T_e$  at the separatrix.

In figure 5, the widest scans of the connection length for both AUG and TCV are shown, completing the collisionality scan (see equation (5)). For AUG (figures 5(a)–(c)), only the toroidal magnetic field could be varied from  $-2.5$  T to  $-3.0$  T, leading to an increase in the connection length from approximately 62 m to 75 m. This variation in the magnetic field causes a slight change in both  $T_i$  (figure 5(a)) and  $T_e$  (figure 5(b)), while their ratio remains stable at around 2.4, consistent with the density scan. However, the change in  $L$  is relatively small, which may explain why no corresponding variation in  $T_i/T_e$  is observed. A similar trend is seen in the TCV  $L$ -scan (figures 5(a)–(c)). In this case, the variation of current results in an increase in  $L$  from 17 m to 24 m. Despite this more significant change,  $T_i/T_e$  remains around 2.8 with no noticeable effect. This again indicates that ion–electron coupling is unlikely to be the primary factor shaping the behavior of  $T_i/T_e$  at the separatrix in AUG and TCV.

Figure 6 summarizes the entire collisionality scan in one figure by showing  $T_i/T_e$  as a function of  $\nu_{\text{SOL}}^*$  and  $n_e \cdot L / (T_i^{5/4} \cdot T_e^{3/4})$ . The plot shows data from AUG in red and from TCV in

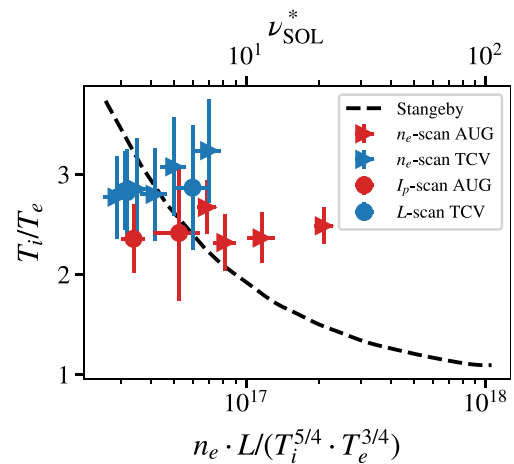


**Figure 5.** Edge ion temperature ((a), (d)), electron temperature ((b), (e)), and electron density ((c), (f)) for an L-mode connection length scan in AUG (from (a) to (c)) and TCV (from (d) to (f)). Different colors represent different connection lengths.

blue. For both datasets, triangles correspond to the  $n_e$ -scan, while circles indicate the  $I_p$ - ( $L$ )-scan. The dashed black line illustrates the indicative trend of  $T_i/T_e$ , as predicted by modelling results shown in figure 4.20 of [7]. While the error bars on the measurements are large and the collisionality scan covers only a portion of Stangeby's plot, the ratio  $T_i/T_e$  at the separatrix does not appear to be affected by collisionality. Hence, the model assuming that the ion heat flux is mostly exhausted through the electrons does not seem to apply in these cases.

## 2.2. Power scan

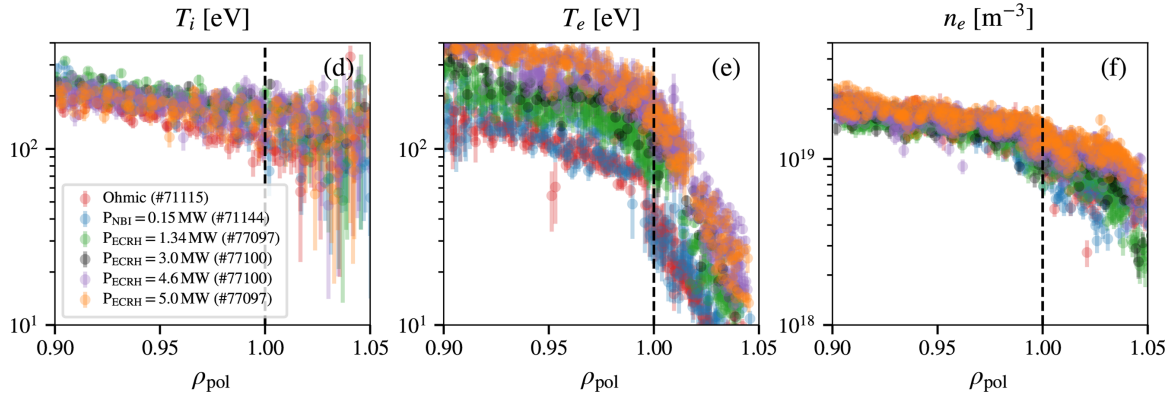
The operational space of ASDEX Upgrade in L-mode with additional heating power is quite limited, as H-mode is triggered very quickly once power is increased. In this experimental study, we could only add 0.4 MW of ECRH, which essentially replaced the ohmic power, as its injection led to a corresponding decrease in ohmic heating. Higher ECRH levels, however, immediately triggered H-mode, making a meaningful comparison with ohmic L-mode discharges impossible. In contrast, TCV allowed for a much wider scan of the heat flux ratio, primarily by adding ECRH in low-density plasmas, where the plasma typically remains in L-mode. The profiles corresponding to increasing ECRH power are extracted from discharges #77097 and #77100. Only one power step was possible with NBI since, as in AUG, higher power would have immediately triggered the L–H transition. The edge profiles from the TCV power scans are shown in figure 7:



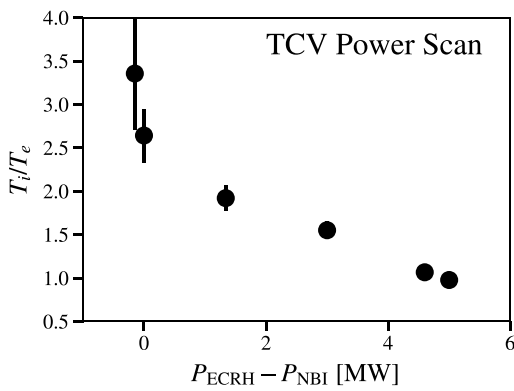
**Figure 6.** Separatrix ion-to-electron temperature ratio as a function of SOL collisionality. AUG data points are shown in red, with triangles indicating the  $n_e$ -scan and circles the  $I_p$ -scan. The same symbols are used for TCV data, shown in blue. The dashed black line represents the indicative trend of  $T_i/T_e$  from modelling results shown in figure 4.20 of [7].

figure 7(a) shows  $T_i$ , figure 7(b) shows  $T_e$  and figure 7(c) shows  $n_e$ . The different colors represent the various heating methods and power levels.

Given the increasing electron heating in this power scan, most of the effect is reflected in the electron temperature, while the ion temperature shows relatively small changes. To



**Figure 7.** Edge ion temperature (a), electron temperature (d), and electron density (c) for an L-mode power scan in TCV using both NBI and ECRH heating. Different colors represent the different heating methods and power level.



**Figure 8.** Ion-to-electron temperature ratio for the TCV power scan as a function of the external input power. Since in TCV the NBI partially heats ions and the ECRH heats electrons, the  $x$ -axis is arranged with positive values for externally added ECRH power and negative values for NBI power, with zero indicating purely ohmic heating.

summarize the effect of the power scan, figure 8 shows the ratio of  $T_i$  to  $T_e$  as a function of the external input power. Since NBI partially heats the ions and ECRH heats electrons in TCV, the  $x$ -axis is ordered with positive values for externally added ECRH power and negative values for NBI power, with zero representing purely ohmic heating. Notably, a substantial amount of electron heating—about 5 MW—is needed to drive  $T_i/T_e$  to 1, whereas ohmic heating typically provides only a few hundred kW. This additional electron heating pushes  $T_i/T_e$  toward 1, underscoring the importance of  $Q_i/Q_e$  as the primary regulator of the separatrix ion-to-electron temperature ratio.

### 2.3. Heat flux limiting factor calculations

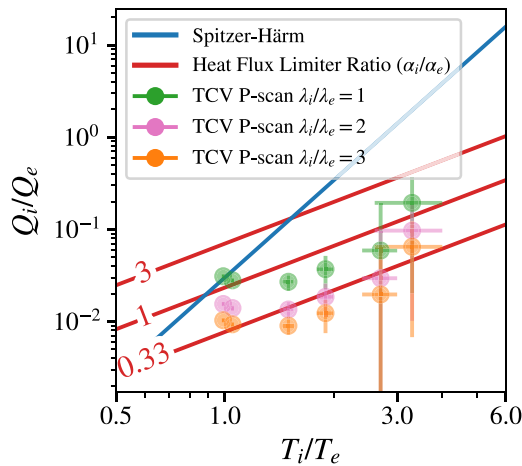
Since collisional heat exchange in the SOL between ions and electrons does not seem to play a major role in determining  $T_i/T_e$  at the separatrix, the simplified model for parallel heat transport described in the first part of section 1

can be applied. By determining the integrated ion-to-electron heat flux ratio through interpretative transport simulations, e.g. using ASTRA [29] as done in this work,  $Q_i/Q_e$  can be plotted as a function of  $T_i/T_e$ . This allows for an assessment of whether the experiment hit the heat flux limiter or are at the Spitzer–Härm heat flux levels. Such comparison is most meaningful for the TCV power scan, where  $Q_i/Q_e$  and thus  $T_i/T_e$  has been varied considerably through the ECRH heating scan. It is important to note that the quantities  $Q_i$  and  $Q_e$ , provided by ASTRA, represent the total core perpendicular heat flux. However, as discussed in the introduction, what needs to be computed and plotted in figure 1 is the integrated parallel heat flux. According to [7, 30], this can be calculated as:

$$Q_{\parallel,(i,e)} = \frac{Q_{\perp,(i,e)}}{2\pi R_u \lambda_{T(i,e)} \sin \theta_{\text{pitch},u}} \quad (7)$$

where  $R_u$  is the upstream major radius,  $\lambda_{T(i,e)}$  is the SOL decay length, and  $\sin \theta_{\text{pitch},u}$  is the sine of the upstream pitch angle. When taking the ratio between  $Q_{\parallel,i}$  and  $Q_{\parallel,e}$ , all other terms cancel out, leaving only  $\lambda_e/\lambda_i$ . This ratio is then used to normalize the ASTRA output.

The results of this analysis are presented in figure 9. The Spitzer–Härm limit is shown in blue, while three different heat flux limiter ratios, 0.33, 1.0, and 3.0, are plotted in red. Experimental data from the TCV power scan are shown in green, pink, and orange, corresponding to three different values of  $\lambda_i/\lambda_e$ , namely 1, 2, and 3. It should be noted that a direct experimental determination of  $\lambda_i$  was not possible, as the CXRS data in the TCV SOL are, as previously mentioned, unreliable. The error bars on  $Q_i/Q_e$  indicate the maximum and minimum variations of its value based on transport simulations using combinations of  $T_i \pm \sigma_{T_i}$  and  $T_e \pm \sigma_{T_e}$  from the experiments. The points on the right-hand side correspond to those on the left-hand side of figure 8. As shown in figure 9, the experimental points align more towards the heat flux limiter limit for an  $\alpha_i/\alpha_e$  ratio roughly between 1.0 and 0.3, depending also on  $\lambda_i/\lambda_e$ . This is roughly consistent with what is assumed in SOL and divertor simulations and marks the first experimental verification of this assumption.



**Figure 9.** Integrated ion-to-electron heat flux ratio as a function of the separatrix  $T_i/T_e$ . The Spitzer-Härm limit is shown in blue, three different heat flux limiter ratios in red, and experimental points from the TCV power scan in green, pink, and orange, corresponding to three different values of  $\lambda_i/\lambda_e$ , namely 1, 2, and 3.

### 3. Conclusions and outlook

This study presents a comprehensive investigation into the ion-to-electron temperature ratio ( $T_i/T_e$ ) at the separatrix in the ASDEX Upgrade and TCV tokamaks. Despite expectations,  $T_i/T_e$  showed weak or no dependence on upstream electron density and connection length, suggesting that collisional coupling between ions and electrons in the SOL is not the dominant mechanism regulating  $T_i/T_e$  at the separatrix. Instead, the  $T_i/T_e$  ratio was found to be primarily influenced by the relative ion and electron heat fluxes from the core, as increased electron heating in TCV through ECRH drove  $T_i/T_e$  toward unity. This suggests that core heating balance, not SOL collisionality, is the key factor governing the separatrix temperature ratio. Since collisional coupling is negligible, plotting  $Q_i/Q_e$  as a function of  $T_i/T_e$  shows that the experimental data align well with the heat flux limiter model for an  $\alpha_i/\alpha_e$  ratio between 0.3 and 1.0, consistent with values used in SOL and divertor simulations. This marks the first experimental confirmation of this result.

To build on these findings, dedicated fluid simulations with EMC3-EIRENE are currently underway to reproduce the experimental results and further refine the understanding of the underlying mechanisms regulating the separatrix ion-to-electron temperature ratio, as for the example the effect of the heat flux limiter. Full-f gyrokinetic simulations would be the next logical step to gain deeper insight into the kinetic part of the parallel transport. In parallel, experiments have already been performed on MAST-U, and data analysis is currently in progress, which will provide an opportunity to test the generality of these results across different tokamak configurations. Moreover, future work will extend this investigation to H-mode plasmas, where the higher confinement and altered

edge conditions could reveal new insights into the role of core heating and SOL transport in shaping  $T_i/T_e$ .

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